

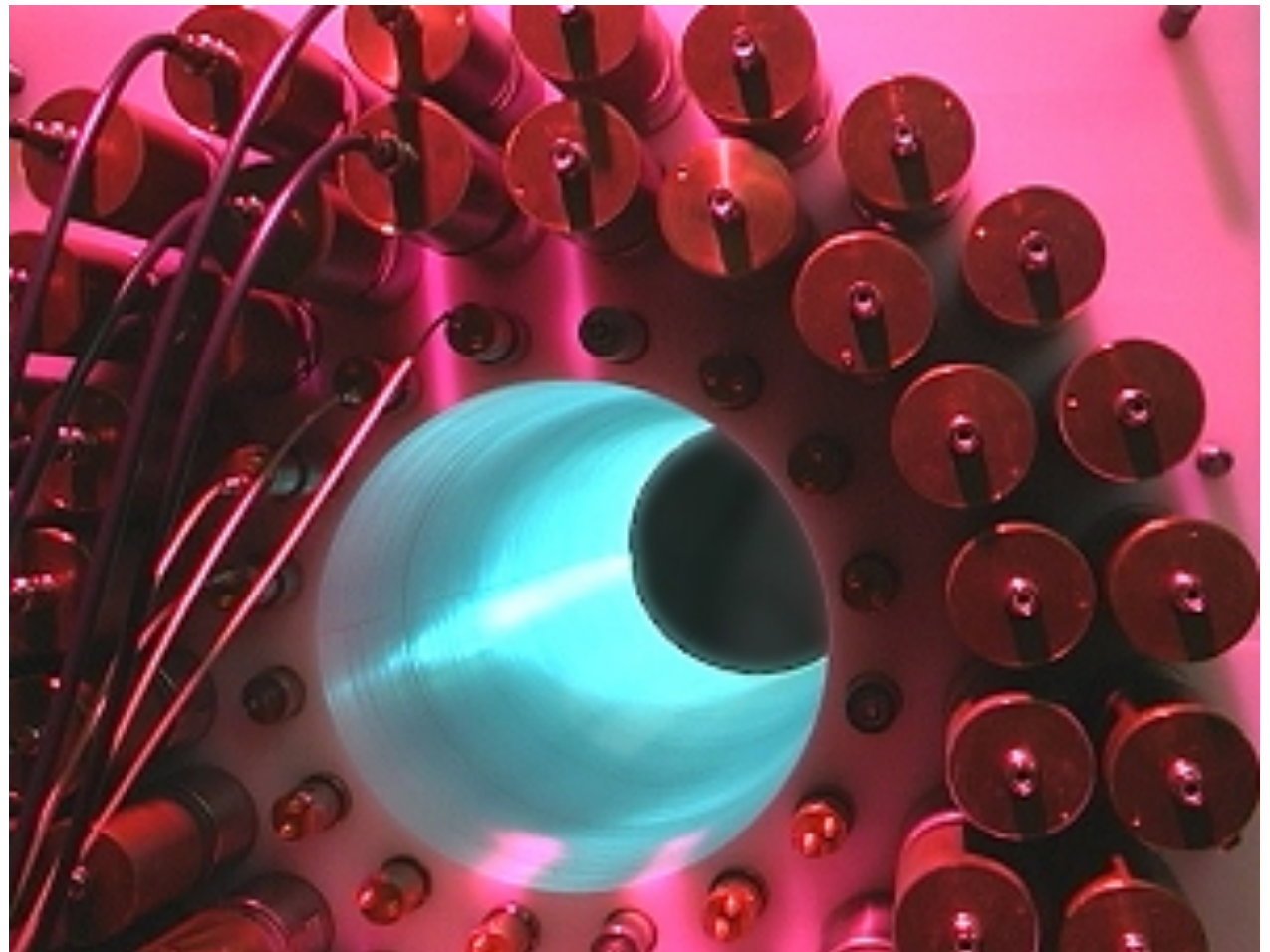
Week 10: Chap. 14 Slow Neutron Detection

(Gamma ray) Backgrounds

Slow Neutron Detection

- nuclear reactions
- spectra properties
- Proportional Counters
- fill gas
- lined detectors

Fast neutron detection



Chap. 14 – Slow Neutron Detection

All neutron detection relies on observing a neutron-induced nuclear reaction.

The nuclear cross sections have a characteristic variation with energy:

- Charged particle reactions are dominated by the coulomb energy since both reaction partners have a positive charge: $\sigma(E) \sim \pi r^2 (1 - V/E_{CMS})$ where “V” is the coulomb barrier.

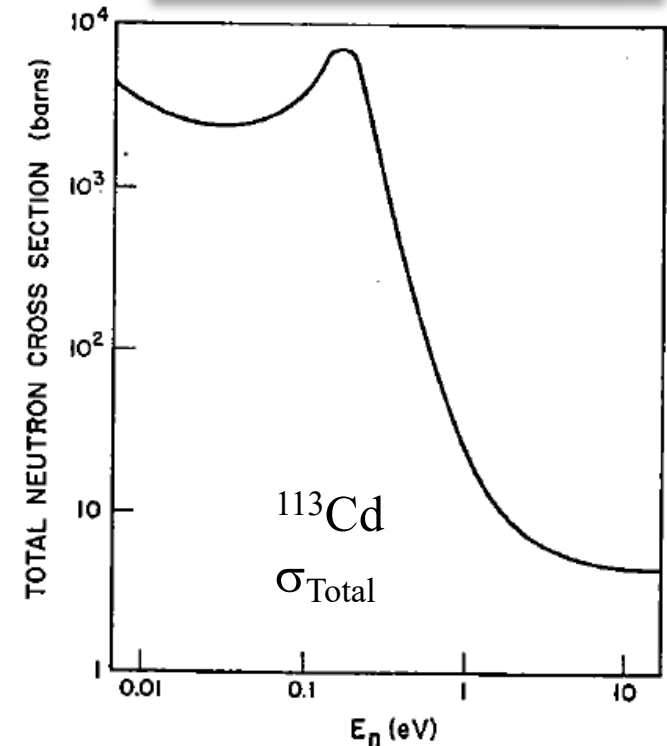
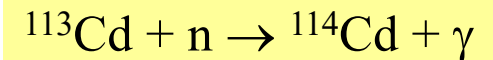
- Neutron-induced reactions also have a characteristic shape .. The interaction is always attractive and the cross section for $l=0$ capture reactions always grows with $1/v$ at (very) low energies. The form is derived from the Breit-Wigner lineshape:

$$\sigma_{Cap} = \pi \hat{\lambda}^2 \frac{\Gamma_n \Gamma_\gamma}{(E - E_0)^2 + (\Gamma / 2)^2}$$

$$\sigma_0 = \pi \hat{\lambda}^2 \frac{4\Gamma_n \Gamma_\gamma}{\Gamma^2} \quad E = E_0; \Gamma = \Gamma_n + \Gamma_\gamma + \dots \cong \Gamma_\gamma$$

$$\sigma_0 = 4\pi \hat{\lambda}^2 \frac{\Gamma_n}{\Gamma_\gamma} \quad \hat{\lambda} = \frac{\hbar}{mv} \quad \Gamma_n \sim v$$

$$\sigma_0 \sim \frac{1}{v}$$



'Popular' Slow Neutron Reactions

$n + {}^3\text{He} \rightarrow ({}^4\text{He})^* \rightarrow p + {}^3\text{H}$, $Q=0.765$ MeV, target abundance = 1.4×10^{-4} % (n,p)

$n + {}^6\text{Li} \rightarrow ({}^7\text{Li})^* \rightarrow {}^4\text{He} + {}^3\text{H}$, $Q=4.78$ MeV, target abundance = 7.5% (n, α)

$n + {}^{10}\text{B} \rightarrow ({}^{11}\text{B})^* \rightarrow {}^7\text{Li}^* + {}^4\text{He}$, $Q=2.31$ MeV, Branch=94%, target abundance = 19.9% (n, α)
 $\rightarrow {}^7\text{Li} + {}^4\text{He}$, $Q=2.79$ MeV, Branch=6%

$n + {}^{113}\text{Cd} \rightarrow ({}^{114}\text{Cd})^* \rightarrow {}^{114}\text{Cd} + \gamma$, $Q \sim 8$ MeV, target abundance = 12.2% (21k barns) (n, γ)

$n + {}^{157}\text{Gd} \rightarrow ({}^{158}\text{Gd})^* \rightarrow {}^{158}\text{Gd} + \gamma$, $Q \sim 8$ MeV, target abundance = 15.6% (255k barns) (n, γ)

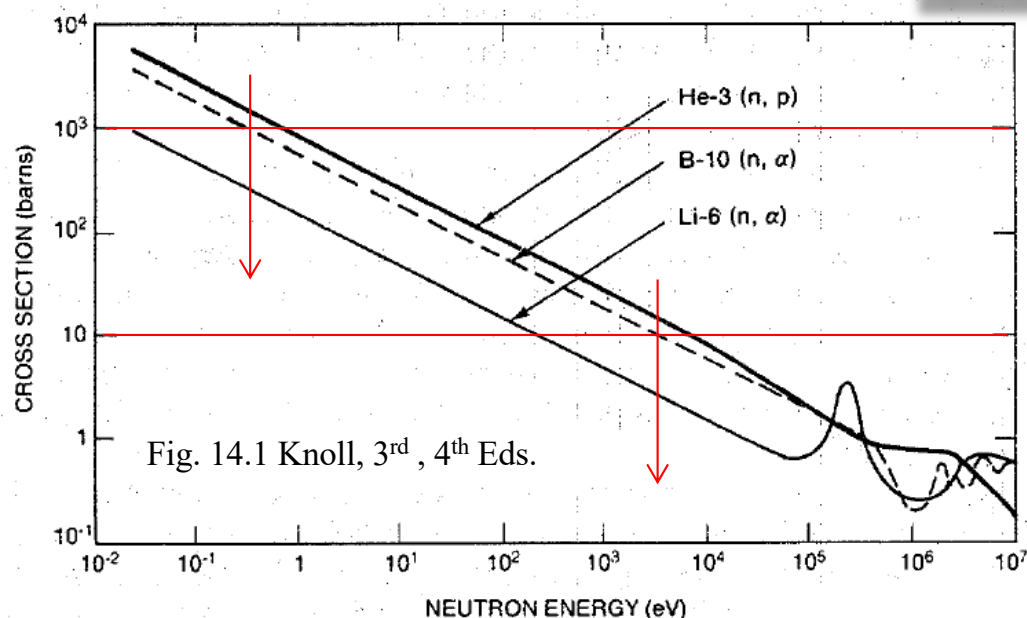
$n + {}^{235}\text{U} \rightarrow ({}^{236}\text{U})^* \rightarrow (\text{fission frags})$ $Q \sim 200$ MeV, TKE ~ 160 MeV, abundance = 0.72% (n,f)

Check:

$$y = ax + b$$

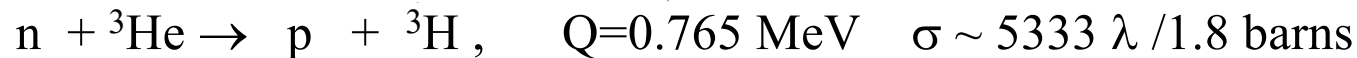
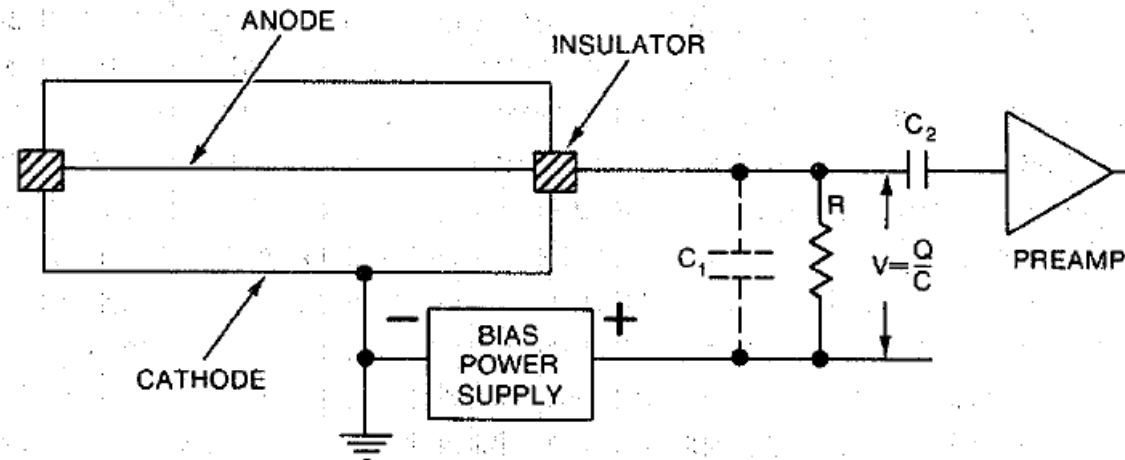
$$\text{Log} \sigma = \left(\frac{\Delta y}{\Delta x} \right) \text{Log} E + b$$

$$\frac{\Delta y}{\Delta x} = -0.485 \rightarrow \sigma = E^{-0.485} \sim 1/\sqrt{E}$$



Slow n Detection: Gas-filled counters

Gas-filled proportional counter .. usual gas-gain amplification on the central anode



K.E. <eV ~0 → Q3/4 Q/4
 0.573 0.191 MeV

Range in Si ~6μm ~5μm

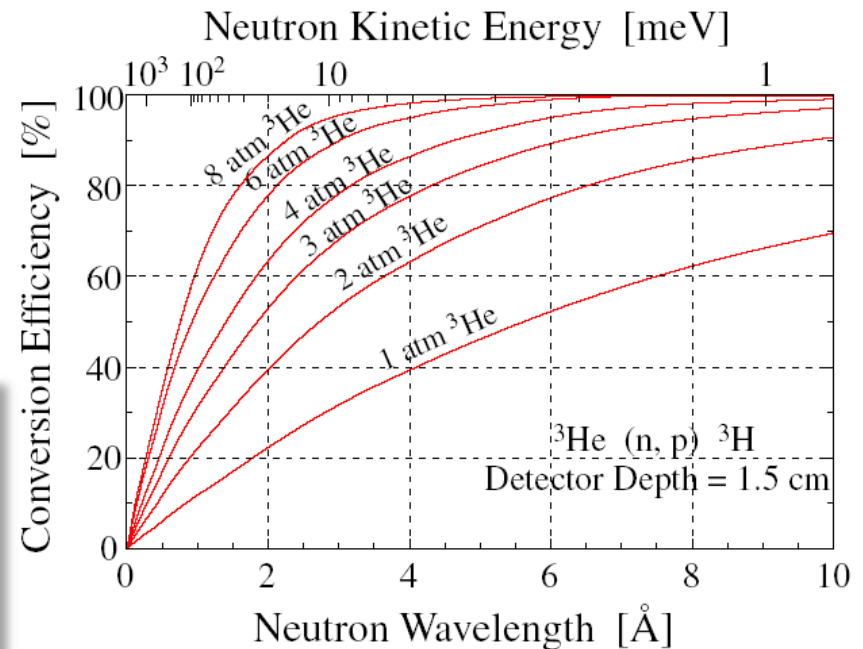
Range in gas ~ 10³ x Range in solid ... few mm's
 [N.B. (Rρ) ~ 0.25 mg/cm² for these α's in He gas]

What about the pulse-height distribution?

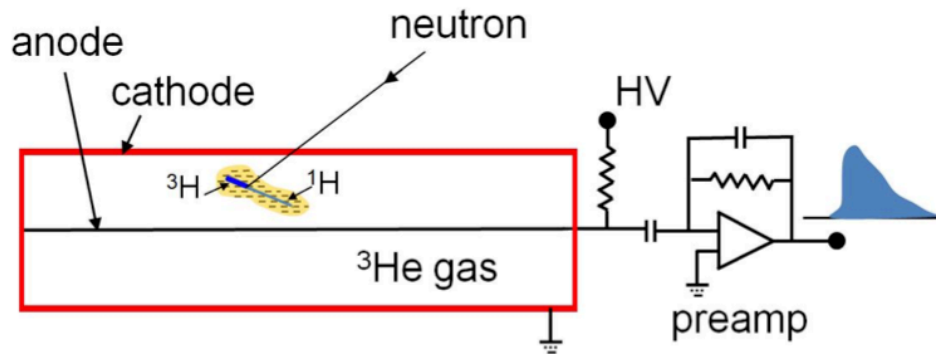
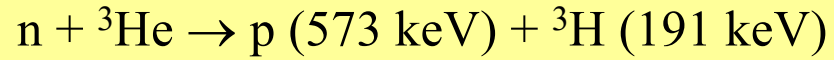
Reaction in gas volume:

→ both products range out -- full energy signal

Reaction elsewhere → ??



Slow n Detection: Gas-filled Pulse Height



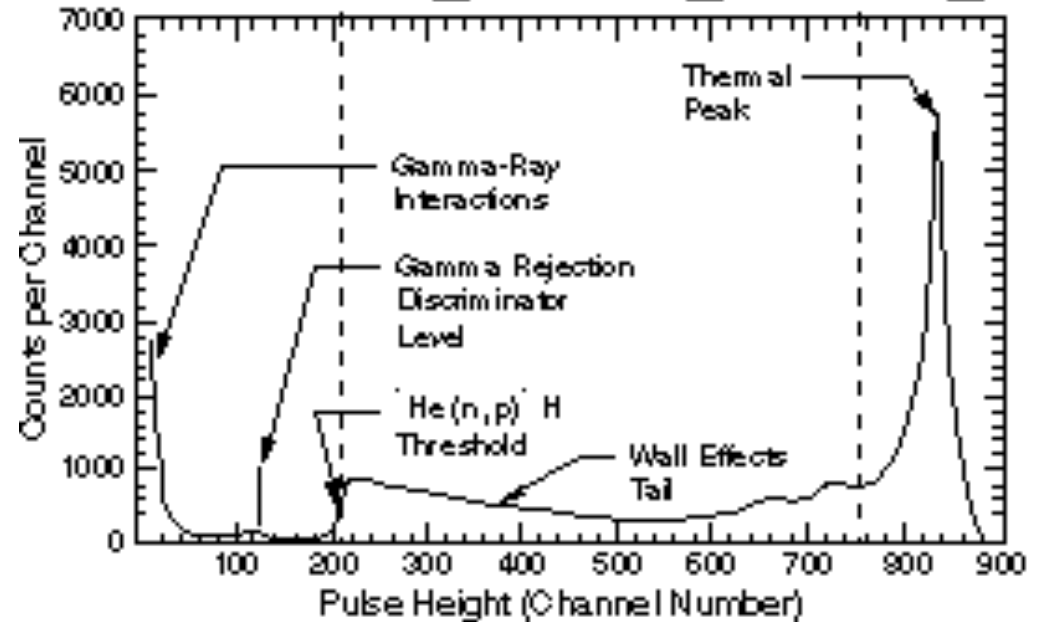
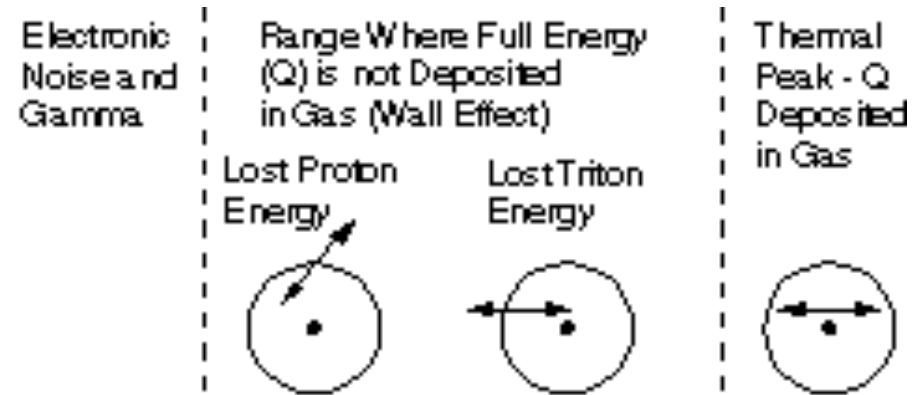
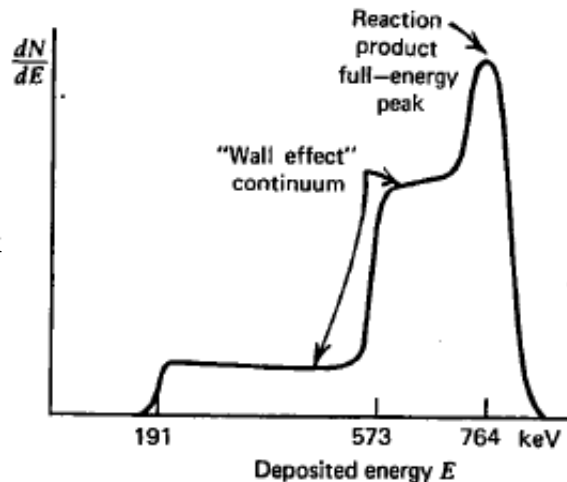
${}^3\text{He}$ gas $W \sim 33 \text{ eV}$

Typical: 25mm diameter, 50 μm anode

$P = 5\text{-}10 \text{ bar (!)}$

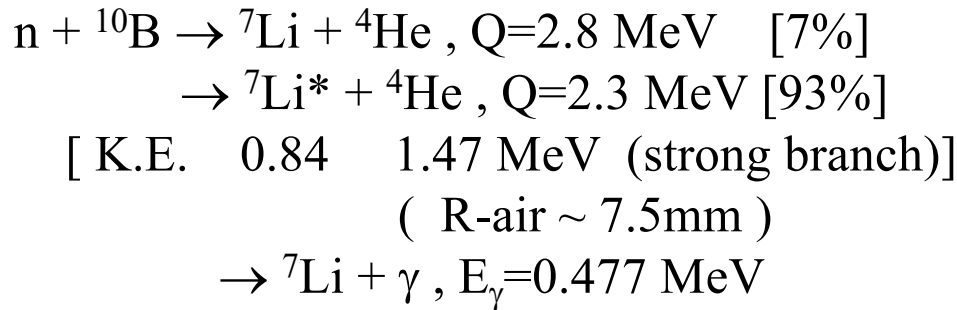
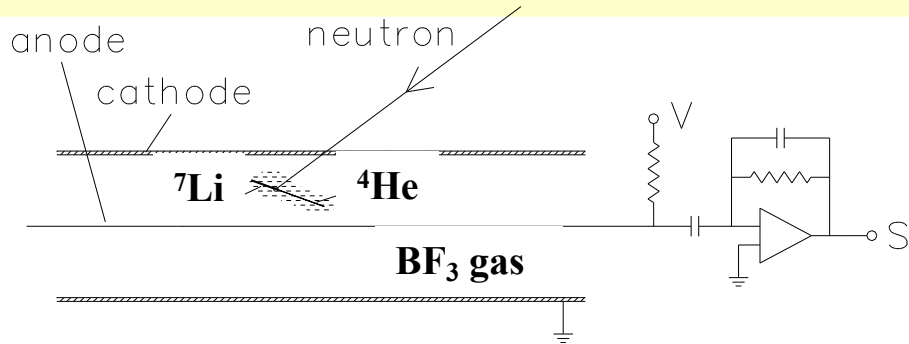
$V \sim 1.5\text{kV} \dots M \sim 20 \dots C \sim 20\text{pF}$

Ideal



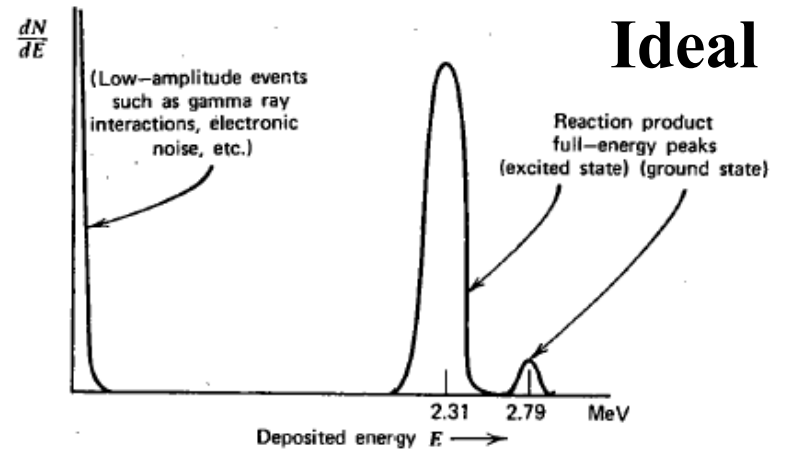
e.g. Rad.Meas. **47** (2012) 577
Adv.Spa.Res. **49** (2012) 1670

Slow n Detection: BF₃

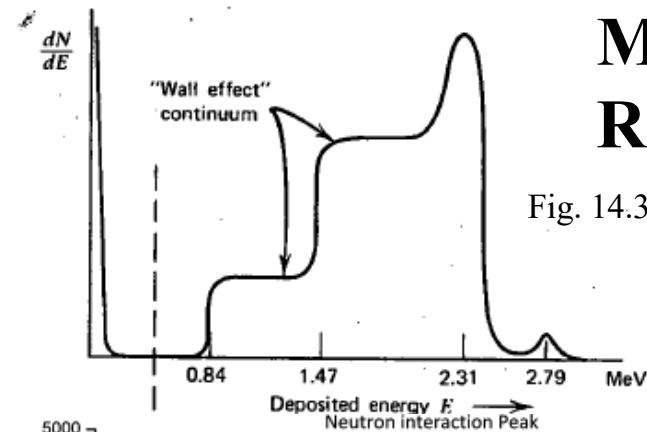


BF₃ gas W ~ 33 eV, enriched to ~96%
 Typical: 25mm diameter, 50μm anode
 P = 400 – 700 Torr, (nero 900 Torr)
 V ~ 2.5kV .. M ~ 40

Annl Nucl Energy 62 (2013) 1

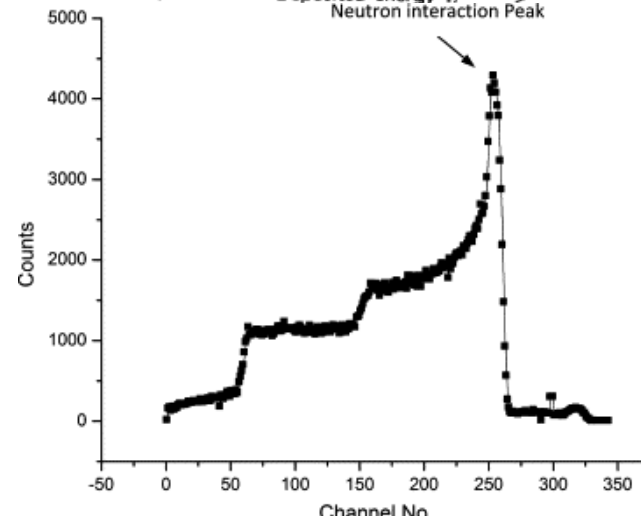


Ideal



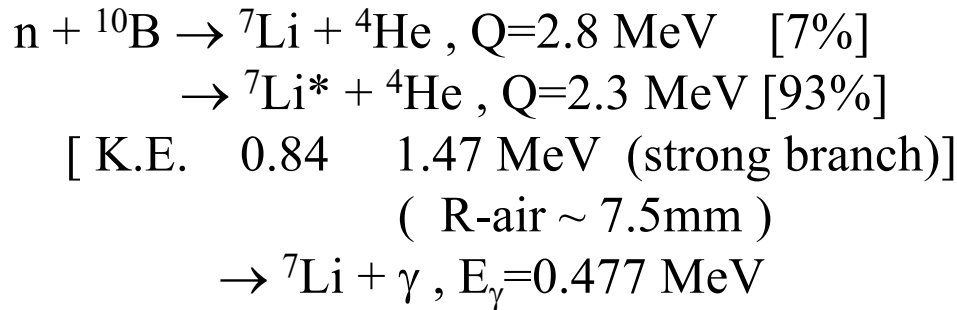
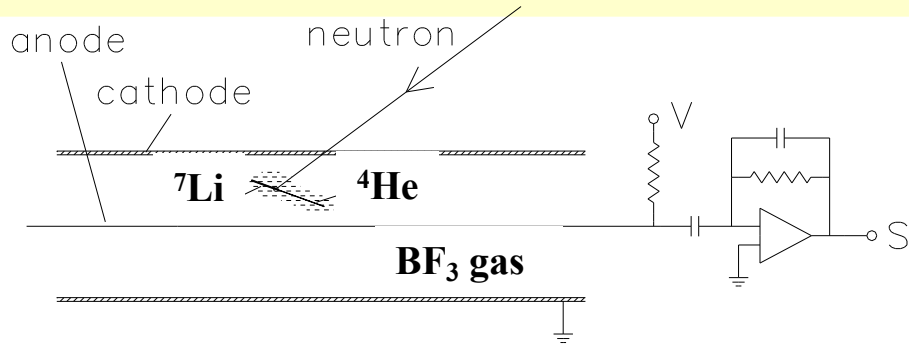
More Realistic

Fig. 14.3 Knoll, 3rd, 4th Eds.



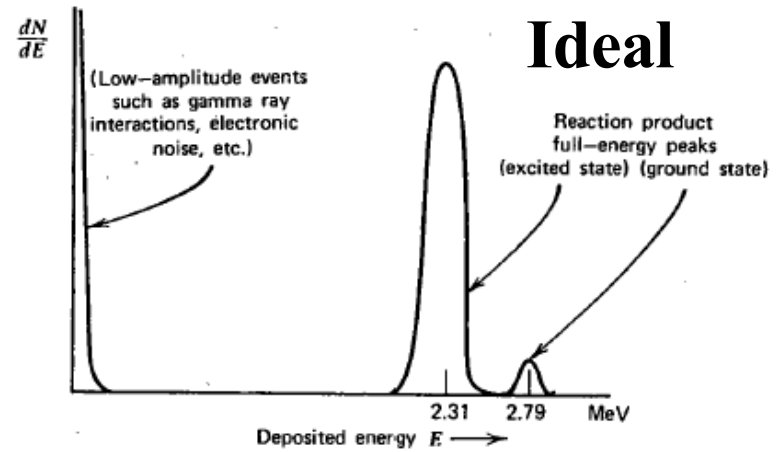
Real

Slow n Detection: BF₃

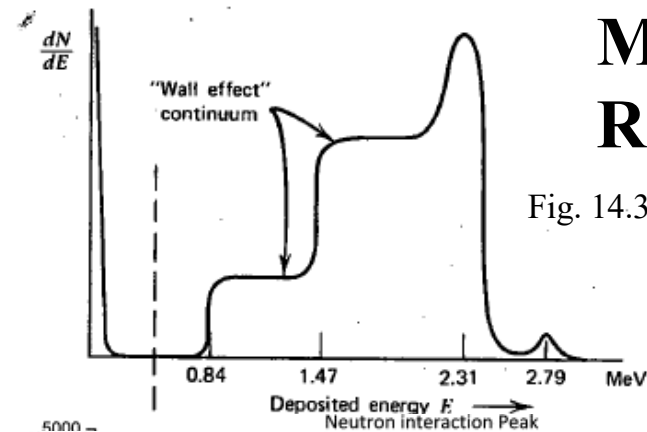


BF₃ gas W ~ 33 eV, enriched to ~96%
 Typical: 25mm diameter, 50μm anode
 P = 400 – 700 Torr, (nero 900 Torr)
 V ~ 2.5kV .. M ~ 40

NIMA 623 (2010) 1035

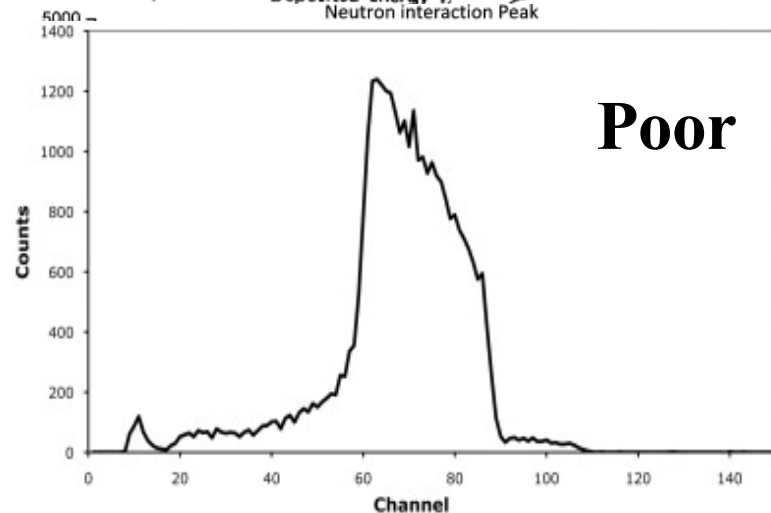


Ideal



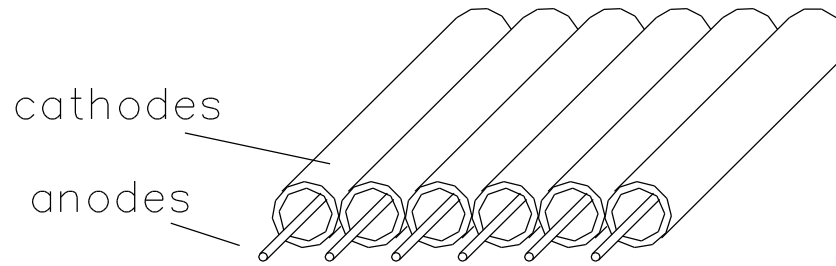
More Realistic

Fig. 14.3 Knoll, 3rd, 4th Eds.

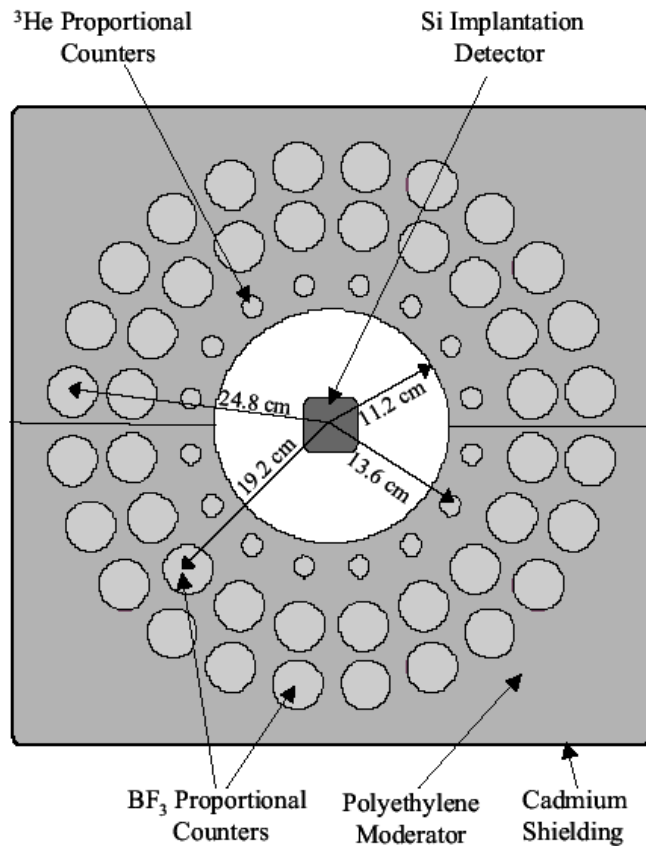


Poor

Multiple Gas Proportional Counters

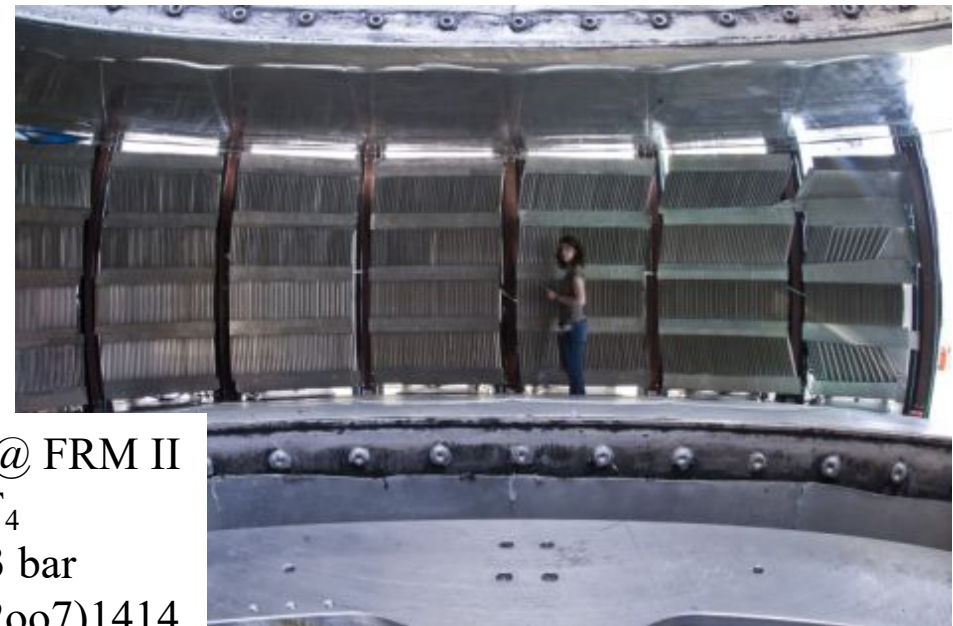
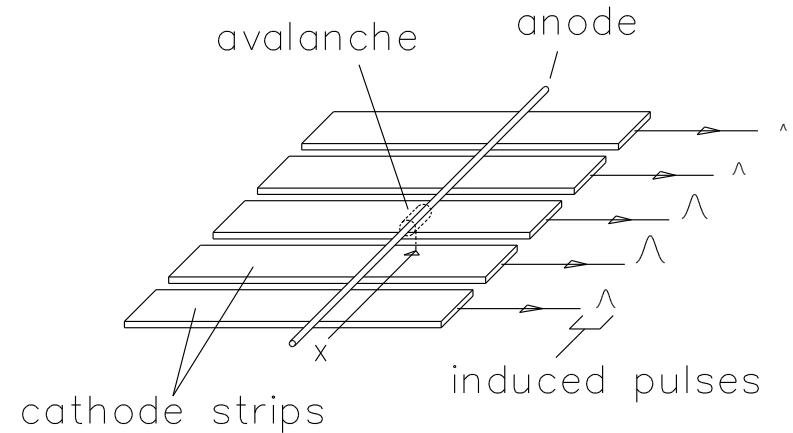


High Efficiency



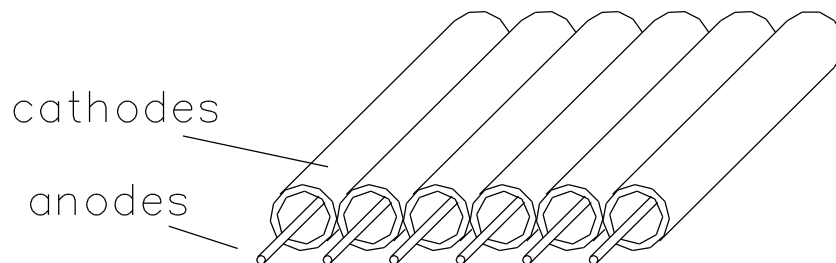
“NERO” @ NSCL
NIM A618 (2o1o) 275

Position Sensitive

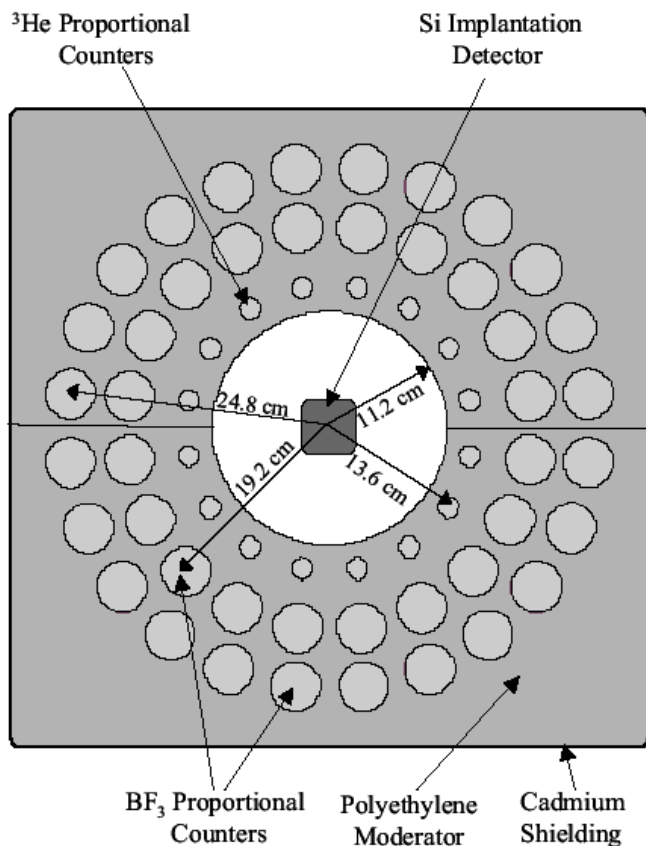


“TOFTOF” @ FRM II
1000 $^3\text{He}/\text{CF}_4$
9.7/0.3 bar
NIM A580(2oo7)1414

Multiple Gas Proportional Counters

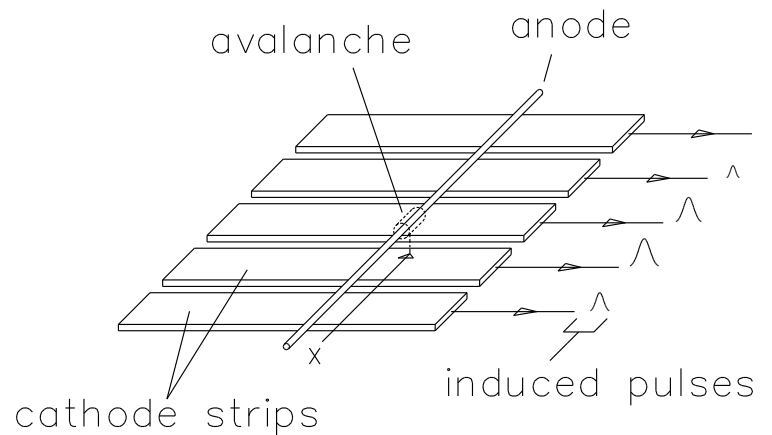


High Efficiency

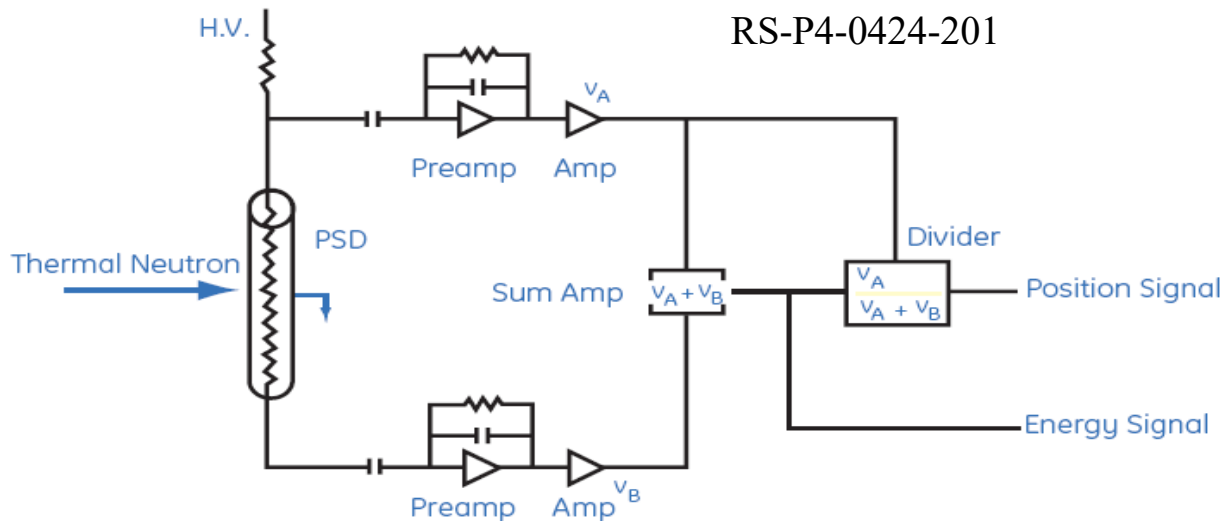


“NERO” @ NSCL
NIM A618 (2o1o) 275

Position Sensitive

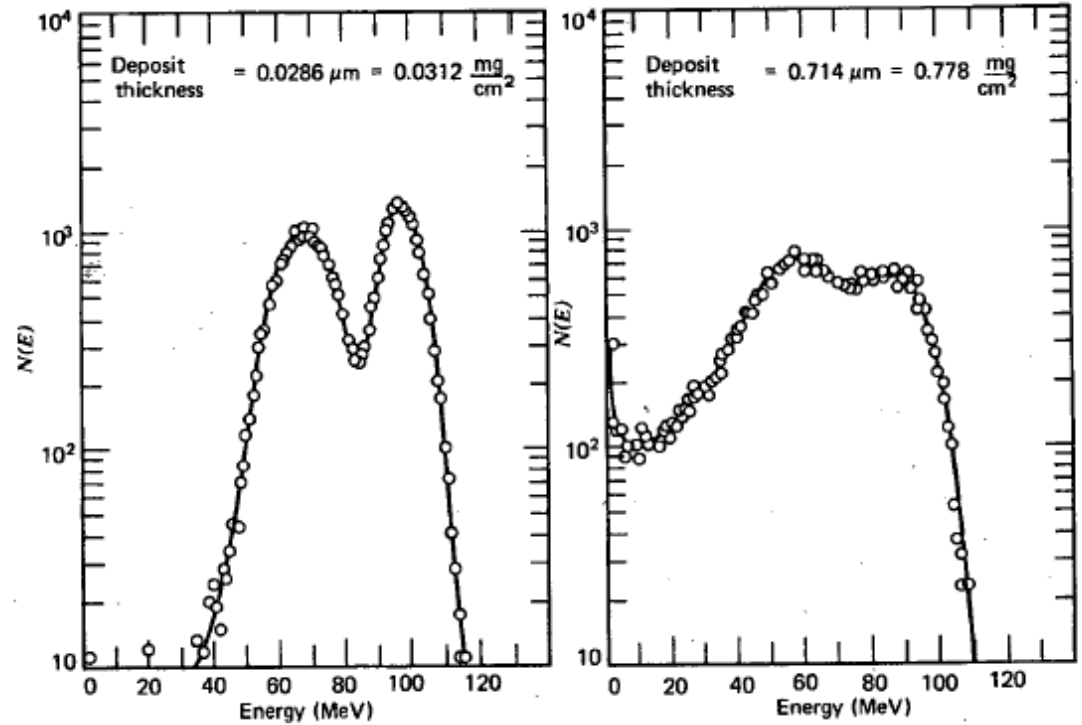
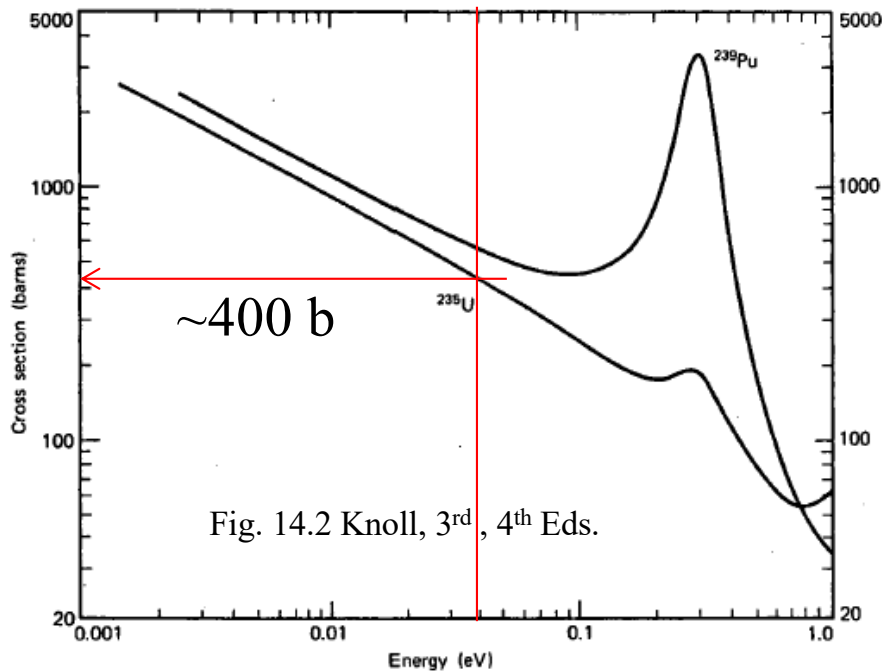


General Electric
RS-P4-0424-201



Slow n Detection: Fission Chambers

$n + {}^{235}\text{U} \rightarrow ({}^{236}\text{U})^* \rightarrow (\text{fission frags})$ $Q \sim 200\text{MeV}$, $\text{TKE} \sim 160\text{MeV}$, abundance = 0.72%



UO_2 deposit on wall Change by $\sim 25x$

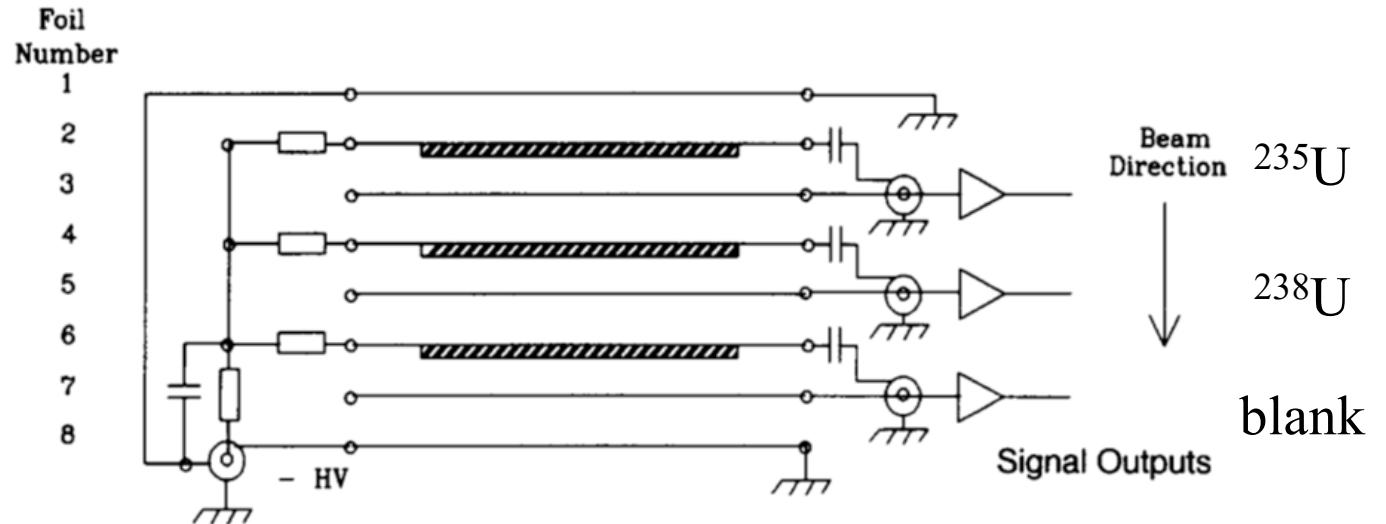
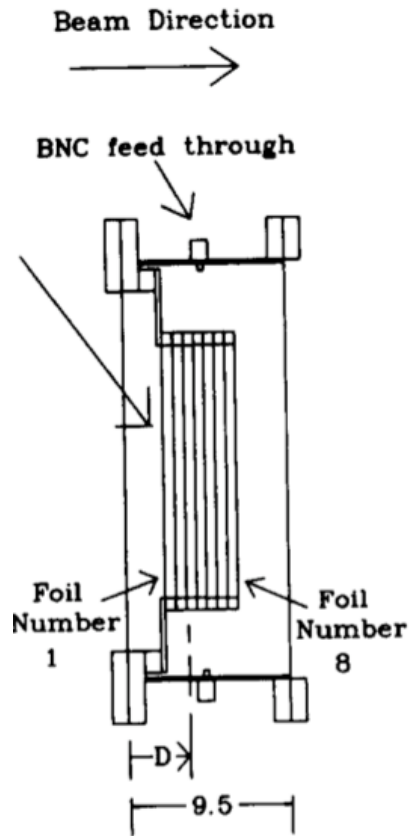
Fig. 14.7 Knoll, 3rd, 14.8 4th Eds.

Absorption efficiency of all nuclear reaction-based devices (from text):

$$\varepsilon(E_n) = 1 - e^{-\Sigma(E)\Delta x} \text{ where } \Sigma(E) = \rho_N \sigma(E)$$

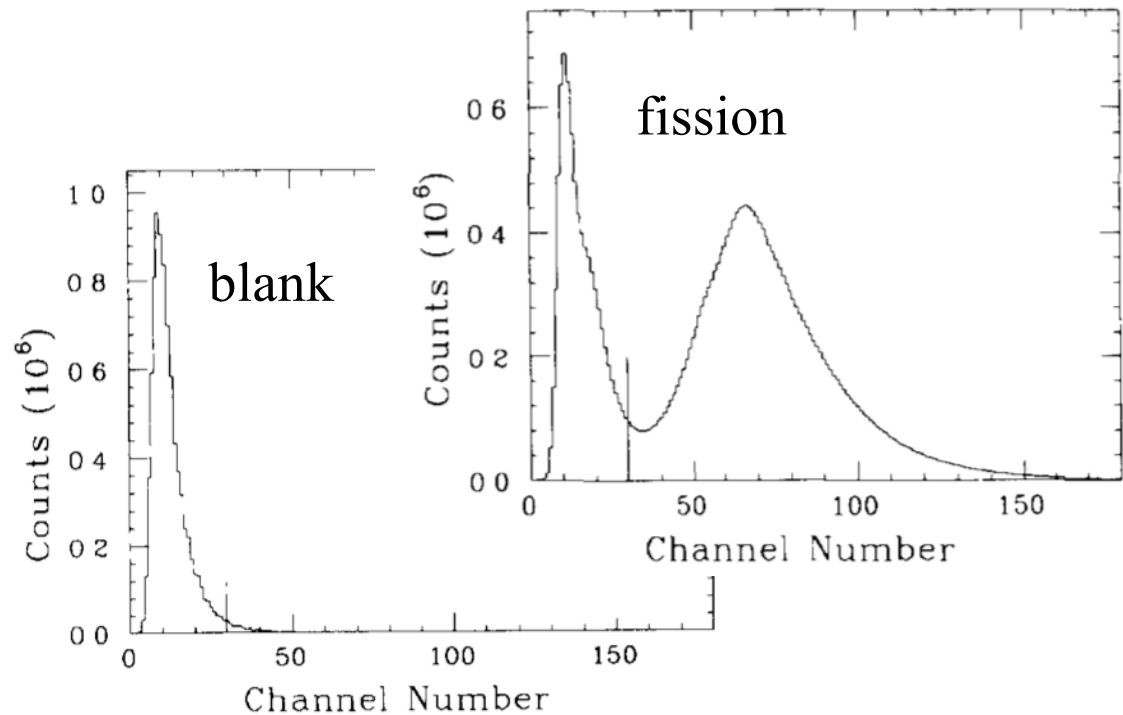
$$\varepsilon(E_n) \sim \rho_N \sigma(E) \Delta x \text{ for small values}$$

Slow n Detection: Fission Chambers

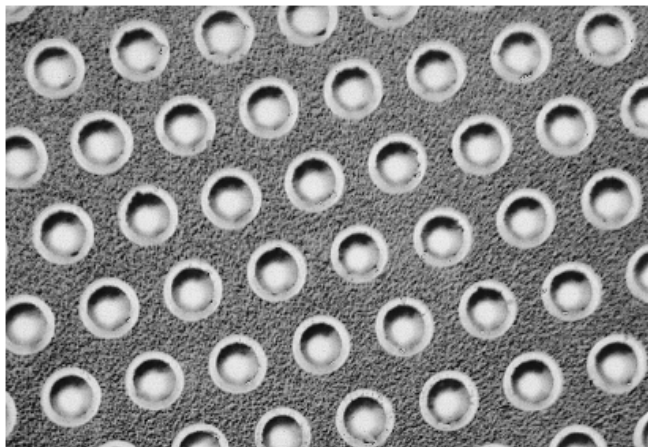
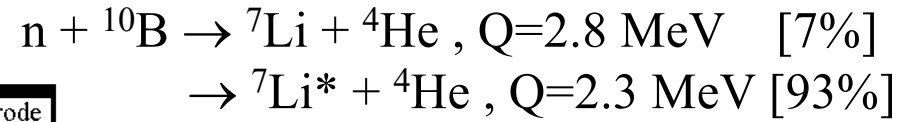
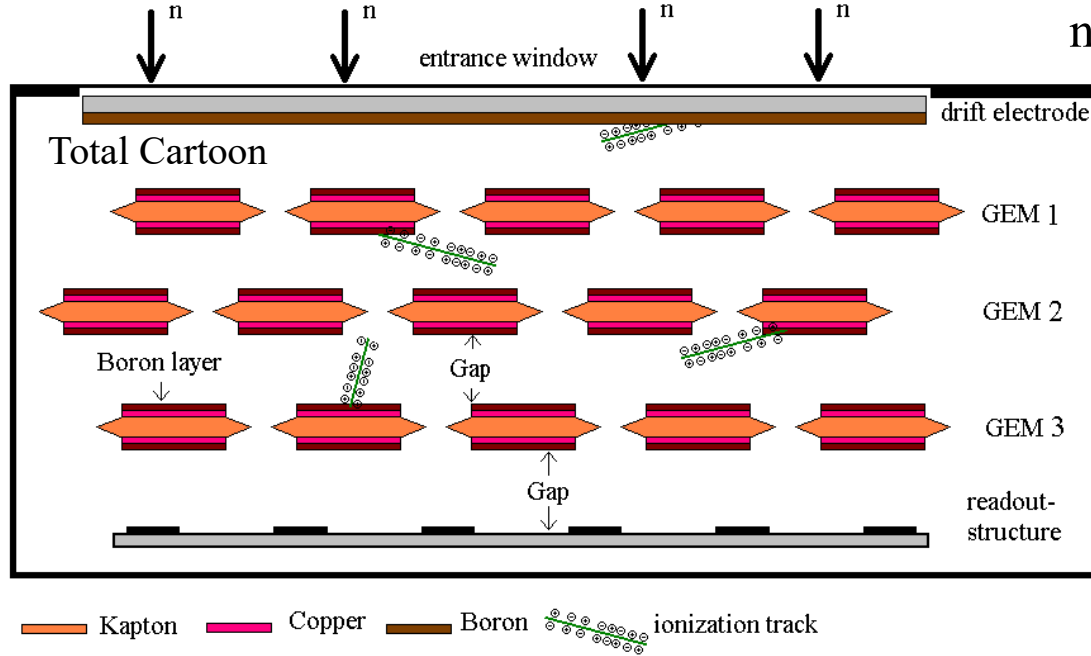


Beam Monitor @ LAMPF
NIM A336 (1993) 226

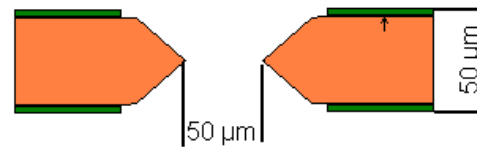
68 kPa gauge pressure P-10
Parallel plates, 6mm gap, -300 V



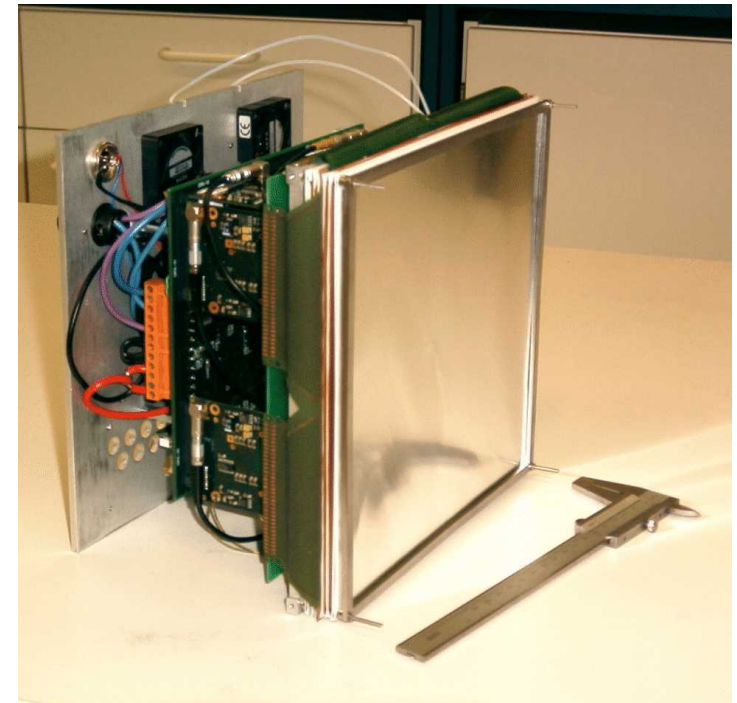
Boron-loaded GEM device



GEM-hole



— Kapton — Copper



CASCADE Detector, 200 mm x 200 mm, stack of GEM-foils with 2D-readout-structure in the middle, complete readout and histogramming electronics on the backside of the detector

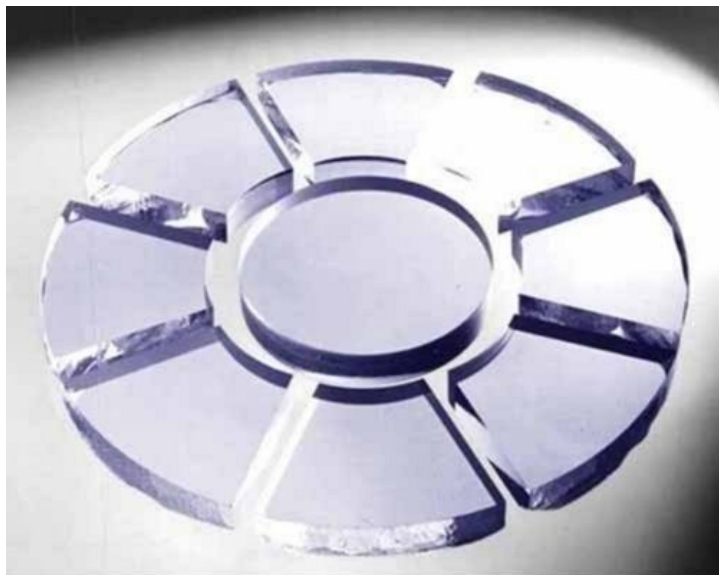
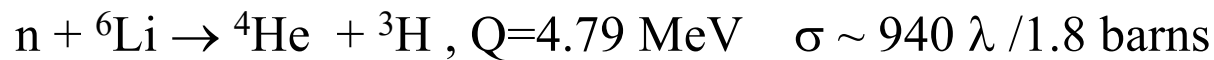
<http://www.physi.uni-heidelberg.de/Forschung/ANP/Cascade/>

Slow n Detection: lithium scintillators

Lithium loaded materials .. usual scintillation device with PMT for ToF

1) Li glass .. Scintillation efficiency $\sim 0.45\%$ 395nm, with $\sim 7k$ photons/n
4% of NaI(Tl), current manufacturers quote 15%

2) LiI (Eu) .. 2.8% 470 nm with $\sim 51k$ photons/n [\sim same as NaI(Tl)]

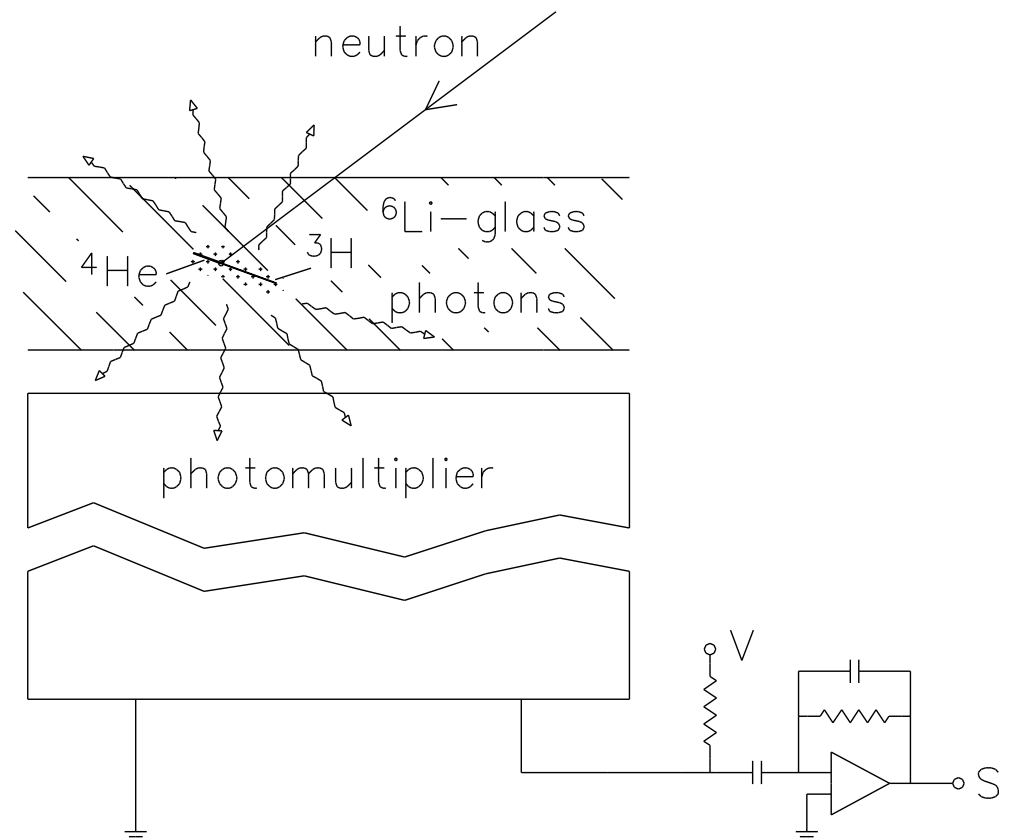


Lithium Glass Array for neutron detection, developed and manufactured by Levy Hill Laboratories for AWRE. Dimensions 8 1/2" by 1/2" thick.

Use Wayback Machine to see:

http://www.apace-science.com/ast/g_scint.htm

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Chap. 14 – Slow n Detection Question

Estimate the intrinsic efficiency and signal height from a fission chamber for thermal neutrons made up as follows: a natural uranium metal coating of 1 mg/cm^2 on the inside of a 1cm diameter tube with a $50\mu\text{m}$ central anode. The tube is filled with Ar/Methane (P-10) at 1 atm pressure and is operated at 1000V and has a (stray) capacitance of 50 pF.

$$\varepsilon(E_n)$$

